

Scheme of GIG Experimental Mine 'Barbara' devoted to natural background radiation (NBR) characterization

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Tables







1. General information

Name of Underground Laboratory: GIG Experimental Mine 'Barbara' (https://www.gig.eu/pl/kd-barbara)

Localization (country/city): Poland/Mikołów

Coordinates to the facility: 50° 10' 47.1" North 18° 55' 58.6" East

Altitude of the facility: about 287 m



Name of the responsible scientist/measurer: University of Silesia, Poland: Jan Kisiel Agata Walencik-Łata Katarzyna Szkliniarz

National Centre for Nuclear Research, Poland: Karol Jędrzejczak Jacek Szabelski

Place where the data is stored (e.g. file in a drawer X, internal data cloud etc.): pendrive, external memory,







2. Measurements of the NBR in Underground Laboratory

Description of the sites where the in-situ measurements were performed:

Hall ID	Dimension of the	Air volume	Depth below	Environmental condition				
	cavern [height x length	exchange	surface	(average temperature				
	x width] (m)	rate /	[m w.e.]	/average humidity)				
		Ventilation						
		[m³/s]						
blind	2.65 x 30 x 3	30-40	122	12 °C / 70%				
chamber								
excavation								

Tab. 1 Description of the sites where the in-situ measurements were performed.



Fig. 1 Scheme of the workings where the in-situ measurements were performed.





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a. In-situ gamma-ray measurement

Information about measurement

The in-situ measurement of the natural background radiation (NBR) was performed by using portable gamma-ray spectroscopy and HPGe (high-purity germanium) detector. For spectra registration and analysis, the multichannel analyzer and special software packages were applied. Energy calibration (covering the range of 14 – 2506 keV) was performed using a set of sealed radioactive sources (1 cm in diameter and about 40 kBq activity): ¹³³Ba, ¹³⁷Cs, ⁵⁴Mn, ⁵⁷Co, ¹⁰⁹Cd, ²²Na and ⁶⁰Co. The efficiency calibration was performed with the use of a geometry of room/box with internal surface contamination, modelled in Geometry Composer software applying ISOCSTM software and monoenergetic photons covering the energy range of 10 – 3300 keV (an adequate measuring range). The monoenergetic photon flux density was estimated from the photopeak areas employing detection efficiency curve $\varepsilon(E)$, Ge crystal surface area and lifetime of spectrum acquisition. An effective dose was estimated using photon flux-to-dose conversion coefficients for isotropic irradiation with monoenergetic photons, interpolated on the basis of data provided by the International Commission on Radiological Protection (ICRP) in report 116 using a third-degree Lagrange formula. Identification of in-situ registered radioisotopes was based on the photopeaks' energies.

Measurements performed in free space

Efficiency modeling

Horizontal orientation of spectrometer

Geometry for efficiency calibration was based on room/box with internal surface contamination (software template) with dimensions of investigated localization. Radioactivity was assumed to be distributed equally between 5 out of 6 walls.



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Fig. 2 ⊤he geometry for modeling the detection efficiency, obtained with the use of Geometry Composer v.4.2.1 (Canberra Industries, Inc.).

Description of detector settings during measurement (where the detector was placed (near wall, floor....):

detector position (horizontal, vertical): horizontal position

distance from the walls: 30 cm

measurement of the gamma-ray spectrum from the wall, floor, concrete, other: **30 cm in front of the nearest wall (the centre of the germanium crystal was 25 cm above the floor)** use of a collimator during the measurement (if yes, what): **no**







• <u>Results</u>

Tab. 2 Results of in-situ gamma-ray measurements.

Hall ID (and place)	Measurement method	Equipment type	Detection relative efficiency [%]	Energy range [keV]	Results (integrated counts per second) [cps]
blind chamber excavation	Gamma spectroscopy	-GR4020 portable spectrometer, -HPGe coaxial detector, -InSpector™ 2000 multichannel -analyser (for data collecting), -Genie™ 2000 v.3.2.1 software package (for spectra analysing)	40	7-3150	350.00±0.05*

*data are presented in A. Walencik-Łata, et al., Characteristics of Natural Background Radiation in the GIG Experimental Mine 'Barbara', Poland. Energies 2022, 15, 685

Schematic plan of the Hall with marked sites where the measurements were performed.



Fig. 3 Schematic plan of the blind chamber excavation with the marked site where the in-situ gamma-ray measurement was performed.











Fig. 4 Gamma-ray spectrum from a blind chamber excavation site.

Other relevant information:

Tab.	3 Results of	gamma-ray fl	ux, gamm	na-ray dose,	and contri	bution o	f the	radioisotopes	for the	investigated	localizations.

Hall ID	Gamma flux	Gamma-ray dose	Radioisotopes that have the main				
(and place)	[cm ⁻² s ⁻¹]	[µSv/h]	contributions ineffective dose				
			Decay chain	Isotope	Concentration		
					[%]		
blind	17.6 ± 1.9*	0.200 ± 0.029*	-	⁴⁰ K	46.7		
chamber			Uranium	²¹⁴ Bi	25.1		
excavation			Thorium	²⁰⁸ TI	15.2		
			Thorium	²²⁸ Ac	6.6		
			Uranium	²¹⁴ Pb	3.3		

*data are presented in A. Walencik-Łata, et al., Characteristics of Natural Background Radiation in the GIG Experimental Mine 'Barbara', Poland. Energies 2022, 15, 685







Qualitative analysis of gamma-ray spectra registered in underground laboratory.

Qualitative analysis of gamma ray spectra registered in underground laboratory (LT - live time, measuring time; Area – net area of the photopeak on the spectrum gamma ray) (SEP – single escape peak, DEP – double escape peak)

Tab. 4 Qualitative analysis of gamma-ray spectrum registered in EM 'Barbara' mine – blind chamber excavation.

True Energy [keV]	Isotope	CPS = Area/LT	Δ CPS
12.97	X: Th L _{α1}	0.2643	0.0054
46.5	²¹⁰ Pb	0.2510	0.0099
51.62	²³⁹ Pu	0.1364	0.0106
74.97	X:Pb K _{α1}	1.1055	0.0297
77.12	X:Bi K _{α1}	2.3414	0.0349
87.34	X:Bi K _{β1}	0.8424	0.0156
92.59	²³⁴ Th	0.3343	0.0133
129.07	²²⁸ Ac	0.1179	0.0151
186.1	²²⁶ Ra	0.2052	0.0131
209.25	²²⁸ Ac	0.1019	0.0111
238.63	²¹² Pb	1.1381	0.0072
241.98	²¹⁴ Pb	0.6409	0.0061
258.87	²¹⁴ Pb	0.0448	0.0096
270.24	²²⁸ Ac	0.1196	0.0054
277.35	²⁰⁸ TI	0.0661	0.0049
295.21	²¹⁴ Pb	1.0995	0.0057
300.09	²¹² Pb	0.0968	0.0037
327.6	²²⁸ Ac	0.0673	0.0072
338.32	²²⁸ Ac	0.2643	0.0088
351.92	²¹⁴ Pb	1.9431	0.0098
387.0	²¹⁴ Bi	0.0279	0.0069
409.46	²²⁸ Ac	0.0299	0.0058
438.83	⁴⁰ K (DEP)	0.0163	0.0057
463.01	²²⁸ Ac	0.0996	0.0062
480.43	²¹⁴ Pb	0.0093	0.0023
487.08	²¹⁴ Pb	0.0135	0.0025
510.77	²⁰⁸ TI	0.2703	0.0064
562.5	²²⁸ Ac	0.0170	0.0046
583.19	²⁰⁸ TI	0.5853	0.0065
609.31	²¹⁴ Bi	1.6776	0.0074
665.45	²¹⁴ Bi	0.0474	0.0022





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719.86 214 Bi0.01420.0019727.2 212 Bi0.12910.0026755.32 228 Ac0.01210.0044768.36 214 Bi0.16780.0055785.91 214 Pb0.05200.0020794.95 228 Ac0.07520.0021806.17 214 Bi0.04130.0019
727.2212Bi0.12910.0026755.32228Ac0.01210.0044768.36214Bi0.16780.0055785.91214Pb0.05200.0020794.95228Ac0.07520.0021806.17214Bi0.04130.0019
755.32228 Ac0.01210.0044768.36214 Bi0.16780.0055785.91214 Pb0.05200.0020794.95228 Ac0.07520.0021806.17214 Bi0.04130.0019
768.36214Bi0.16780.0055785.91214Pb0.05200.0020794.95228Ac0.07520.0021806.17214Bi0.04130.0019
785.91 ²¹⁴ Pb 0.0520 0.0020 794.95 ²²⁸ Ac 0.0752 0.0021 806.17 ²¹⁴ Bi 0.0413 0.0019
794.95 ²²⁸ Ac 0.0752 0.0021 806.17 ²¹⁴ Bi 0.0413 0.0019
806.17 ²¹⁴ Bi 0.0413 0.0019
835./1 ^{22°} Ac 0.0210 0.0019
839.03 ²¹⁴ Pb 0.0280 0.0021
860.56 ²⁰⁸ TI 0.0771 0.0046
911.21 ²²⁸ Ac 0.4369 0.0056
934.06 ²¹⁴ Bi 0.1020 0.0048
949.83 ⁴⁰ K (SEP) 0.0311 0.0042
964.77 ²²⁸ Ac 0.0947 0.0020
968.97 ²²⁸ Ac 0.2595 0.0027
1001.03 ²³⁴ Pa 0.0093 0.0029
1051.96 ²¹⁴ Bi 0.0115 0.0037
1069.96 ²¹⁴ Bi 0.0084 0.0015
1078.62 ²¹² Bi 0.0089 0.0015
1120.29 ²¹⁴ Bi 0.4043 0.0029
1133.66 ²¹⁴ Bi 0.0038 0.0016
1155.19 ²¹⁴ Bi 0.0482 0.0039
1238.11 ²¹⁴ Bi 0.1641 0.0048
1253.5 ²¹⁴ Bi (SEP) 0.0046 0.0029
1280.98 ²¹⁴ Bi 0.0339 0.0036
1377.67 ²¹⁴ Bi 0.0980 0.0016
1385.31 ²¹⁴ Bi 0.0197 0.0010
1401.5 ²¹⁴ Bi 0.0336 0.0012
1407.98 ²¹⁴ Bi 0.0618 0.0014
1460.83 ⁴⁰ K 4.2242 0.0077
1495.9 ²²⁸ Ac 0.0127 0.0008
1501.577 ²²⁸ Ac 0.0078 0.0008
1509.23 ²¹⁴ Bi 0.0555 0.0012
1538.5 ²¹⁴ Bi 0.0138 0.0009
1543.32 ²¹⁴ Bi 0.0069 0.0007
1580.54 ²²⁸ Ac 0.0224 0.0009
1588.21 ²²⁸ Ac 0.0506 0.0012
1592.53 ²⁰⁸ TI (DEP) 0.0420 0.0011
1599.31 ²¹⁴ Bi 0.0092 0.0007
1620.5 ²¹² Bi 0.0220 0.0008
1630.63 ²²⁸ Ac 0.0255 0.0008

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1638.28	²²⁸ Ac	0.0088	0.0007
1661.28	²¹⁴ Bi	0.0259	0.0018
1683.99	²¹⁴ Bi	0.0053	0.0006
1693.21	²¹⁴ Bi (DEP)	0.0073	0.0007
1729.6	²¹⁴ Bi	0.0669	0.0011
1764.5	²¹⁴ Bi	0.3440	0.0026
1838.36	²¹⁴ Bi	0.0079	0.0006
1847.42	²¹⁴ Bi	0.0426	0.0009
1873.16	²¹⁴ Bi	0.0087	0.0015
2052.94	²¹⁴ Bi	0.0031	0.0009
2103.5	²⁰⁸ TI (SEP)	0.0599	0.0010
2118.55	²¹⁴ Bi	0.0253	0.0008
2204.21	²¹⁴ Bi	0.0957	0.0019
2293.36	²¹⁴ Bi	0.0097	0.0015
2447.86	²¹⁴ Bi	0.0286	0.0013
2614.7	²⁰⁸ TI	0.4453	0.0024
2769.9	²¹⁴ Bi	0.0003	0.0001
2880.3	²¹⁴ Bi	0.0002	0.0001
2978.9	²¹⁴ Bi	0.0003	0.0001

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Apparent radioactivity and effective dose rate calculated on the base of the gamma-ray spectrum.

Tab. 5 Effective dose rate and apparent radioactivity results calculated based on the gamma-ray spectrum for blind chamber excavation site*.

Decay chain	Radionuclide	Effective dose rate [pSv/s]	% contribution	Apparent activity in measuring point [Bq/cm ²]
Uranium	²³⁴ Th	0.025 ± 0.004	0.05	3.36 ± 0.59
Thorium	²²⁸ Ac	3.66 ± 0.54	6.6	1.45 ± 0.40
Uranium	²²⁶ Ra	0.036 ± 0.006	0.06	1.89 ± 0.42
Thorium	²¹² Pb	0.342 ± 0.049	0.61	1.12 ± 0.21
Uranium	²¹⁴ Pb	1.85 ± 0.27	3.32	2.55 ± 0.51
Thorium	²⁰⁸ TI	8.43 ± 1.23	15.16	0.63 ± 0.18
Uranium	²¹⁴ Bi	13.97 ± 2.07	25.13	3.26 ± 0.38
Thorium	²¹² Bi	0.571 ± 0.085	1.03	1.92 ± 0.23
Uranium	^{234m} Pa	0.032 ± 0.011	0.06	1.23 ± 0.40
	⁴⁰ K	26.06 ± 3.70	46.88	51.63 ± 5.10
	²³⁹ Pu	0.011 ± 0.002	0.02	19.00 ± 5.06
Uranium:	²¹⁰ Pb	0.018 ± 0.003	0.03	2.45 ± 0.34
	X rays	0.343 ± 0.049	0.62	







		annihilation	0.242	± 0.035	0.44				
*data ar	e presented in A	A Walencik-Łata	et al	Characteris	tics of Natural	Background	Radiation in	the	GIO

*data are presented in A. Walencik-Łata, et al., Characteristics of Natural Background Radiation in the GIG Experimental Mine 'Barbara', Poland. Energies 2022, 15, 685

b. Measurements of the radon concentration in air

Information about measurement

Measurements of the radon concentration in the air were performed using the RAD7 radon detector. The measurements were made in cycles.

Description of detector settings during measurement (where the detector was placed (near wall, floor....):

Place of the measurement: in blind chamber excavation, detector was located opposite/near the gamma-ray spectrometer – right corner

Ventilation (on/off): ventilation on and off

Other relevant information (use a drying unit, humidity, temperature, pressure, other): measurement was performed with use a drying unit, humidity 70%, temperature 12°C

• <u>Results</u>

Tab. 6 The results of radon concentration in the air at the EM 'Barbara' mine – blind chamber excavation.

Hall ID	Measurement method	Equipment type	Collection period	Average result [Bq/m ³]
		RAD7		205-34300
blind chambor	Alpha spectroscopy	electronic	9 days (0.5	(depending on
		radon	- 1 h-long	the ventilation
excavation		detector	each)	of the
		(Durridge),		chamber)

Schematic plan of the Hall with marked sites where the measurements were performed.









Fig. 5 Schematic plan of the blind chamber excavation site with the marked sites where the measurements of the radon concentration in air were performed.

Plot of radon activity concentration in blind chamber excavation.





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c. Measurements of the neutron flux

Information about measurement

The thermal neutron flux was measured using helium counters, i.e., gas proportional counters filled with helium-3 (³He). In this type of detector, neutrons are captures by helium nucleus in the reaction: ³He(n,p)³H + 764 keV and the charged products (proton and Tritium) of the reaction appear with kinetic energy 764 keV. This method allows distinguishing neutrons events even with a large background of other types of radiation. The setup consisted of a flat tray of helium counters positioned vertically in a rack, four Centronic counters, and four ZDAJ counters. The data acquisition system (DAQ) was designed and made in NCBJ-Łódź laboratory. DAQ is built in the form of a cassette with eight measurement cards. Each card contains four independent measurement channels, up to 32 input channels in total. Each measurement channel is sampled with a frequency of 10 MHZ by an ADC with a dynamic of 10 bits. DAQ contains a built-in Raspberry Pi computer. Therefore it works as an autonomous system, without the need to connect to an external computer. The analyze the pulse shape (PSD) was applied. PSD allows rejecting disturbances. The PSD analysis was based on the two-dimensional distribution of the maximum pulse amplitude versus the maximum pulse jump on the rising edge.

Standard method:

a) Type of counters (helium/boron) and name of the manufacturer, counter's dimensions: proportional ³He counters, Centronic counters (Centronic 50He3/190/50MS), ZDAJ counters (ZDAJ NEM425A50). The counters of both types were steel pipes 50 cm long, but they differed in diameter (5 cm for Centronic and 2.5 cm for ZDAJ) and helium-3 pressure (2 atm for Centronic and 4 atm for ZDAJ).

b) Setup description: number of counters, their distances in a tray etc.:

A flat tray of helium counters was positioned vertically in a rack. The tray was divided into two parts: first, there were four Centronic counters, then there was a 30 cm space, and again four ZDAJ counters. There were 5 cm gaps between the individual counters.

c) Place of measurement: chamber name, distance from walls, large objects nearby and what they are made of (water, polyethylene, graphite, etc.): detection setup for the neutron measurements was placed in the center of blind chamber excavation, a distance few cm from walls
d) Humidity: normal / low / high







• <u>Results</u>

Tab. 7 Results of the neutron flux measurements.

Hall ID	Collection period	Average neutron counting rate	Dispersion of neutron counting rate	Calculated neutron flux witch error
blind	2 weeks	16.6 ± 2.8 (for		(8.6±1.1)·10 ⁻⁶
chamber		Centronic) and		[cm ⁻² s ⁻¹]*
excavation		6.8±1.2 (for		
		ZDAJ) neutrons		
		per hour*		

*data are presented in A. Walencik-Łata, et al., Characteristics of Natural Background Radiation in the GIG Experimental Mine 'Barbara', Poland. Energies 2022, 15, 685

Schematic plan of detector location in the blind chamber excavation.



Fig. 7 Schematic plan of neutron detector located in the blind chamber excavation.







Distribution of amplitude of signals from ³He counter:











3. Laboratory analyses of the water samples

Measurements of the concentration of radium and uranium radioisotopes in water samples were carried out in an external laboratory - "Low-level Activity Research Laboratory", Institute of Physics, the University of Silesia in Katowice, Poland.



Fig. 9 Diagram of the 30 m level, layer 310, where water, sediment, carbon, stone, slate, and sandstone samples were collected for laboratory analysis. Part one.









Fig. 10 Diagram of the 30 m level, layer 310, where water, sediment, carbon, stone, slate, and sandstone samples were collected for laboratory analysis. Part two.



Fig. 11 Diagram of the 30 m level, layer 310, where water, sediment, carbon, stone, slate, and sandstone samples were collected for laboratory analysis. Part three.







Description of the sites where water samples were taken:

Tab. 8 Description of the sites where water samples were collected.

Hall ID	Depth below	Sites (wall, water gullets, water	Method of the
	surface	reservoir, other)	sampling
	[m w.e.]		
Localization 1 - water			
sample 1			
Localization 2 - water			
sample 1,2bis			
Localization 3 - water			
sample 3			
Localization 4 - water	80 (lawar 210)	water gullets,	Sampling for
sample 4	80 (layer 310)	stagnant water	polyethylene bottles
Localization 5 - water			
sample 5			
Localization 6 - water			
sample 6			
Localization 8 - water			
sample 8			

The samples were acidified immediately after collection to avoid precipitation of radionuclides and adsorption on the walls of the containers.



Fig. 12 Water sampling site from EM 'Barbara'.



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a. Uranium concentration in water samples

• <u>Description of the performed chemical procedure</u>

The concentration of uranium ²³⁴U and ²³⁸U isotopes in the water samples were determined by semiconductor alpha spectrometry using a 7401VR spectrometer (Canberra, USA) and an Alpha Analyst^M (Mirion Technologies (Canberra), Inc., USA) (**Fig. 13**). A radiochemical procedure was applied to prepare an alpha spectrometric source before measurement. The samples were acidified with HNO₃ and spiked with a known amount of ²³²U. The separation of U is performed using anion exchange resin Dowex 1×8 (Cl- type, 200-400 mesh) based on a procedure worked out by Suomela (1993). The spectrometric source was prepared by co-precipitation of U with NdF₃ and deposition on polypropylene disks (0.1 µm) (Pall Corporation).

References:

J. Suomela, Method for determination of U-isotopes in water, Swedish Radiation Institute, Stockholm, SSI-rapport, 0282-4434, 93:14 (1993).





Fig. 13 (a) the spectrometer 7401VR (Canberra, USA), (b) the alpha spectrometer Alpha Analyst[™] (Mirion Technologies (Canberra), Inc., USA).







• Information about measurement and results

Tab. 9 Results of uranium radioisotopes concentration in water samples from EM 'Barbara'.

Hall ID (name of	Measu-	Equipment	Collection	Limit of	Average	e results	ratio	U [µg/l]
the sample)	rement	type	detection	[mE	[mBq/l]			
	method				²³⁴ U	²³⁸ U		
Water sample loc.1					3.3±0.3	2.4±0.2	1.39±0.16	0.19±0.02
Water sample loc.1,2bis					3.9±0.2	2.3±0.2	1.70±0.18	0.18±0.02
Water sample loc.3		oha ectro- opy USA)	2-7 days	for	35.4±0.9	27.7±0.8	1.28±0.05	2.24±0.06
Water sample loc.4	Alpha spectro-			0.5 l	5.0±0.3	3.7±0.3	1.34±0.12	0.30±0.02
Water sample loc.5	scopy			sample	2.2±0.2	1.0±0.1	2.14±0.35	0.08±0.01
Water sample loc.6				volume	6.2±0.4	4.6±0.4	1.36±0.14	0.37±0.03
Water sample loc.8					1.7±0.1	0.7±0.1	2.54±0.46	0.06±0.01

Alpha spectrum:



Fig. 14 The alpha spectrum of a water sample – EM 'Barbara'- water sample loc.1.









Fig. 15 The alpha spectrum of a water sample – EM 'Barbara'- water sample loc.1,2bis.



Fig. 16 The alpha spectrum of a water sample – EM 'Barbara' - water sample loc.3.









Fig. 17 The alpha spectrum of a water sample – EM 'Barbara' - water sample loc.4.



Fig. 18 The alpha spectrum of a water sample – EM 'Barbara' - water sample loc.5.







Fig. 19 The alpha spectrum of a water sample – EM 'Barbara'- water sample loc.6.





Fig. 20 The alpha spectrum of a water sample – EM 'Barbara'- water sample loc.8.







4. Laboratory analyses of the rock samples

Measurements of the concentration of uranium, radium and potassium radioisotopes in the rock samples (*Fig. 21*) were performed in an external laboratory - "Low-level Activity Research Laboratory", Institute of Physics, the University of Silesia in Katowice, Poland.

Description of the sites where rock samples were taken:

Hall ID	Depth below surface	Sites (wall, floor, brick, concrete,	Method of the sampling	Name of the sample
	[m w.e.]	other)		
blind chamber	177			Sample 1-18
excavation	122			
Localization 1			newly chipped rock samples	Stone sample loc. 1
Localization 2				Stone sample loc. 2
Localization 2	80	Wall		Carbon sample loc. 2
Localization 6	(layer 310)			Slate sample loc. 6
Localization 6				Sandstone sample loc. 6
Localization 7				Carbon sample loc. 7

 Tab. 10 Description of the sites where rock samples were collected.



Fig. 21 (a) place of rock sampling and in-situ measurements in the EM 'Barbara' (sample 1-18), (b) approximate sampling location, (c) diagram of the place where the samples were taken.



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Fig. 22 (a) diagram of the place where the samples were taken (b) localization 1 - stone sampling, (c) localization 2 – stone and carbon sampling.



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Fig. 23 (a) diagram of the place where the samples were taken (b) localization 6 – slate and sandstone sampling, localization 7 – carbon sampling.

a. Radium and potassium concentration in rock samples

Information about measurement

The concentration of radium and potassium isotopes in the rock samples was determined by gamma spectroscopy with the HPGe detector in a lead shield (*Fig. 24a*). The activity of ⁴⁰K was calculated directly from a single 1460.8 keV line. The activity of ²²⁶Ra was calculated as the weighted mean of the values obtained from the ²¹⁴Pb (295.2, 351.9 keV) and ²¹⁴Bi (609.3, 1120.3 keV) isotopes, while ²²⁸Ra activity was calculated from the gamma lines 338.3 keV and 911.1 keV originating from ²²⁸Ac decay. The total duration of a single measurement depended on the sample activity.

Description of the procedure performed before measurements and conditions during measurements (e.g., use of a Marinelli beaker; shielding of the detector; drying, crushing, grinding, mixing of rock; other relevant information): **Before the measurements, the rock sample**

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was dried, crushed, ground, mixed and placed in a Marinelli container (Fig. 24b,c, Fig. 25), which was then sealed and left for one month to achieve secular equilibrium in the thorium and uranium series. The grains diameter after crushing the rock sample were less than 1 mm. Measurements were made in a shielded cover made of lead and copper.



Fig. 24 (a) HPGe detector with shielding, (b) crushed rock sample no. 11, (c) rock sample in Marinelli containers.



Localization 6 – slate sample 6



Fig. 25 Stone sample from localization 1, carbon sample from localization 7, slate and sandstone sample from localization 6.







Tab. 11 Results of radium and potassium concentration in rock, stone, slate, sandstone and carbon samples.

Hall ID (name of sample)	Measu- rement method	Equipment type		u- Equipment relative nt type [%]	Equipment type			Average resul [Bq/kg]	ts
methou		[/0]		²²⁶ Ra	²²⁸ Ra	⁴⁰ K			
Rock sample 1					3.0 days	9.7±0.4*	9.4±0.3*	476±8*	
Rock sample 2				2.8 days	24.6±0.8*	25.8±0.5*	515±8*		
Rock sample 3				2.9 days	12.3±0.4*	10.6±0.3*	453±7*		
Rock sample 4				1.9 days	14.5±0.5*	12.6±0.4*	560±10*		
Rock sample 5				2.1 days	12.9±0.5*	12.8±0.3*	463±8*		
Rock sample 6				4 days	10.4±0.4*	11.1±0.3*	545±9*		
Rock sample 7				2 days	22.6±0.8*	17.3±0.4*	560±10*		
Rock sample 8				2.1 days	9.7±0.4*	11.0±0.4*	466±8*		
Rock sample 9				3 days	14.2±0.8*	11.5±0.3*	535±9*		
Rock sample 10				2.1 days	10.4±0.4*	8.4±0.3*	434±7*		
Rock sample 11				2.7 days	53.7±1.8*	31.5±0.7*	601±10*		
Rock sample 12				3.1 days	14.7±0.5*	14.0±0.6*	557±9*		
Rock sample 13	Gamma			2 days	25.8±0.9*	28.4±0.6*	485±8*		
Rock sample 14	spectro-	detector	20	2.3 days	35.5±1.2*	31.5±0.7*	609±10*		
Rock sample 15	metry	detector		2 days	24.3±0.9*	14.8±0.4*	541±9*		
Rock sample 16				3.9 days	11.8±0.4*	12.3±0.3*	533±8*		
Rock sample 17				1.9 days	8.1±0.3*	7.0±0.3*	521±9*		
Rock sample 18				2.1 days	11.0±0.4*	10.4±0.3*	446±8*		
Stone sample loc.1				2.1 days	64.9±2.4	58.2±1.2	685.5±11.2		
Stone sample loc.2				2.0 days	53.3±2.0	47.2±1.0	813.7±13.0		
Carbon sample	-			4.6 davs	3.0±0.1	1.2±0.1	3.6±0.1		
loc.2									
Slate sample loc.6				2.1 days	61.3±2.3	51.6±1.0	856.6±13.6		
Sandstone sample loc.6				2.2 days	26.2±1.0	23.8±0.5	668.0±10.8		
Carbon sample loc.7				4.0 days	4.3±0.2	1.8±0.1	8.8±0.5		

*data are presented in A. Walencik-Łata, et al., Characteristics of Natural Background Radiation in the GIG Experimental Mine 'Barbara', Poland. Energies 2022, 15, 685

Gamma-ray spectra:



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Fig. 26 Gamma-ray spectrum of a rock sample 1.



Fig. 27 Gamma-ray spectrum of a rock sample 2.



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Fig. 28 Gamma-ray spectrum of a rock sample 3.



Fig. 29 Gamma-ray spectrum of a rock sample 4.



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Fig. 30 Gamma-ray spectrum of a rock sample 5.



Fig. 31 Gamma-ray spectrum of a rock sample 6.



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Fig. 32 Gamma-ray spectrum of a rock sample 7.



Fig. 33 Gamma-ray spectrum of a rock sample 8.



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Fig. 34 Gamma-ray spectrum of a rock sample 9.



Fig. 35 Gamma-ray spectrum of a rock sample 10.



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Fig. 36 Gamma-ray spectrum of a rock sample 11.



Fig. 37 Gamma-ray spectrum of a rock sample 12.



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Fig. 38 Gamma-ray spectrum of a rock sample 13.



Fig. 39 Gamma-ray spectrum of a rock sample 14.



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Fig. 40 Gamma-ray spectrum of a rock sample 15.



Fig. 41 Gamma-ray spectrum of a rock sample 16.



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Fig. 42 Gamma-ray spectrum of a rock sample 17.



Fig. 43 Gamma-ray spectrum of a rock sample 18.



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Fig. 44 Gamma-ray spectrum of a stone sample loc.1.



Fig. 45 Gamma-ray spectrum of a stone sample loc.2.



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Fig. 46 Gamma-ray spectrum of a carbon sample loc.2.



Fig. 47 Gamma-ray spectrum of a slate sample loc.6.



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Fig. 48 Gamma-ray spectrum of a sandstone sample loc.6.



Fig. 49 Gamma-ray spectrum of a carbon sample loc.7.



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- b. Uranium concentration in rock samples
 - <u>Description of the performed chemical procedure</u>

The concentration of uranium ²³⁴U and ²³⁸U isotopes in the rock samples were determined by semiconductor alpha spectrometry and 7401VR (Canberra, USA) and Alpha Analyst^M (Mirion Technologies (Canberra), Inc., USA) spectrometers (*Fig. 50a,b*). A radiochemical procedure was applied to prepare an alpha spectrometric source before measurement. For this purpose, wet mineralization of the rock sample was performed using hot acids: HF, HNO₃, HCl, with H₃BO₃. Uranium was pre-concentrated with iron and co-precipitated at pH 9. The separation of U was performed using the anion exchange resin Dowex 1×8 (Cl⁻ type, 200-400 mesh) based on a procedure worked out by Suomela (1993). The spectrometric source was prepared by coprecipitation of U with NdF₃ and deposition on polypropylene disks (0.1 µm) (Pall Corporation).

References:

J. Suomela, Method for determination of U-isotopes in water, Swedish Radiation Institute, Stockholm, SSI-rapport, 0282-4434, 93:14 (1993).



Fig. 50 (a) The alpha spectrometer 7401VR (Canberra, USA), (b) The alpha spectrometer Alpha Analyst[™] (Mirion Technologies (Canberra), Inc., USA).







• Information about measurement and results

Tab. 12 Results of uranium radioisotopes concentration in rock samples.

Hall ID (name	Measu- rement	Equipment type	JuipmentCollectionLimit ofAverage resultspeperioddetection[Bq/kg]		e results /kg]	ratio ²³⁴ U/ ²³⁸ U	U [ppm]	
of	method				²³⁴ U	²³⁸ U		
Rock sample 1		spectrometers			10.4±0.8*	10.4±0.8*	1.00±0.10*	0.84±0.06
Rock sample 2		7401VR (Canberra–		0.5 mBq/l for both	22.2±1.1*	21.6±1.0*	1.03±0.07*	1.75±0.08
Rock sample 3	Alpha	Packard) and Alpha		^{234,238} U isotopes	14.9±1.2*	14.7±1.1*	1.01±0.11*	1.19±0.09
Rock sample 4	spectro- scopy	Analyst™ (Mirion	1-7 days	and 0.5 l	15.3±1.1*	15.2±1.0*	1.01±0.10*	1.23±0.08
Rock sample 5		Technologies		sample	15.8±1.0*	15.9±1.0*	0.99±0.09*	1.29±0.08
Rock sample 6		(Canberra), Inc., USA)		volume	11.9±1.0*	10.7±0.9*	1.11±0.13*	0.87±0.07

*data are presented in A. Walencik-Łata, et al., Characteristics of Natural Background Radiation in the GIG Experimental Mine 'Barbara', Poland. Energies 2022, 15, 685

Alpha spectra:



Fig. 51 The alpha spectrum of a rock sample 1 – EM 'Barbara'.









Fig. 52 The alpha spectrum of a rock sample 2 – EM 'Barbara'.



Fig. 53 The alpha spectrum of a rock sample 3 – EM 'Barbara'.









Fig. 54 The alpha spectrum of a rock sample 4 – EM 'Barbara'.



Fig. 55 The alpha spectrum of a rock sample 5 – EM 'Barbara'.





Fig. 56 The alpha spectrum of a rock sample 6 – EM 'Barbara'.

c. Neutron activation of the rock sample

Information about measurement and results

Before the measurements, the rock samples were dried, crushed, and mixed. The grain diameter of the crushed rock sample was less than 1 mm. Then the sample was placed in a plastic bag and activated by neutron flux from a ²⁵²Cf source (*Fig. 57b*) by period of one month. Immediately after activation, the gamma-ray spectrum was measured by gamma spectroscopy with a lead-shielded HPGe detector (*Fig. 57a*). Measurements were carried out in several cycles, in short cycles to determine the short-lived isotopes produced during the activation of the sample by neutrons, and in long cycles to determine the long-lived isotopes.











Fig. 57 (a) HPGe detector, (b) source (252Cf) of neutron activation, (c) rock sample after neutron activation for gamma-ray spectrometry.

Information about measurement and results

Hall ID (name of sample)	Measure- ment method	Equipment type	Detection relative efficiency [%]	Collection period	Source of neutron activation	Neutron flux of the source	Activated isotopes
Sample 11	Gamma scpectrometry	HPGe detector	20	Short measurements and long measurements by a few days	californium (²⁵² Cf)	10 ⁵ n/cm²/s	⁵⁷ Ni, ⁴⁶ Sc, ⁵⁶ Mn, ²⁴ Na, ⁴² K, ¹⁴⁰ La

Tab. 13 Neutron activation results of the rock sample from EM 'Barbara'*.

*data are presented in A. Walencik-Łata, et al., Characteristics of Natural Background Radiation in the GIG Experimental Mine 'Barbara', Poland. Energies 2022, 15, 685

Analysis of rock sample, which underwent neutron activation.

Tab. 14 Comparison of counts per second for a rock sample before and after neutron activation of a rock sample.

		Counts per second				
lsotope	Energy [keV]	Rock sample before neutron activation	Rock sample 67 h after neutron activation	Rock sample 101.7 h after neutron activation		
²¹⁰ Pb	46.5	0.01400	0.00810	0.00810		
²¹⁴ Pb	53.2	0.00779				
²³⁰ Th	63.3	0.04613	0.02314	0.02327		



²³⁰ Th	67.7	0.00236	0.00184	0.00271
X Pb K _{α2}	72.8	0.01307	0.00658	0.00684
X Pb K _{α1}	74.97	0.19313	0.08356	0.08316
²²⁷ Th	77.1	0.30181	0.13156	0.12848
Χ Ρο Κ _{α1}	79.3	0.00289	0.00101	0.00173
X Rn K _{α2}	81.1	0.00278	0.00098	0.00223
X Rn K _{α1}	83.8	0.03198	0.01360	0.01529
Х Ві К _{β1}	87.3	0.10002	0.04410	0.0426
Χ Βί Κ _{β2}	89.8	0.05874	0.02391	0.0244
²³⁴ Th	92.7	0.10448	0.05135	0.0507
Χ U Κ _{α1}	98.4	0.00711		
²²⁸ Ac	99.5	0.01388	0.00777	0.0093
X Ra K _{β1}	103		0.00981	0.0039
X Th K _{β1}	105.6	0.01692	0.00896	0.0072
X Th K _{β2}	108.6	0.00729		0.0026
²³⁴ Th	112.8	0.00508	0.00369	0.0015
²¹² Pb	115.2	0.00656	0.00304	0.0024
²²⁸ Ac	129.1	0.02296	0.01127	0.0102
²²⁸ Th	131.3			0.0011
²³⁵ U	143.8	0.01039	0.00423	0.0060
²²⁸ Ac	154	0.01328	0.00640	0.0059
²³⁵ U	163.4	0.00506	0.00206	0.0036
²³⁵ U	186.1	0.12164	0.05612	0.0524
²²⁸ Ac	191.6	0.00196		
²²⁸ Ac	199.4	0.00315		
²³⁵ U	205.3	0.00561	0.00199	0.0018
²²⁸ Ac	209.3	0.03812	0.01733	0.0154
²²⁸ Th	216.5	0.00277		0.0021
²²⁷ Th	236	0.00853	0.00349	0.0029
²¹² Pb	238.6	0.43551	0.18229	0.1754
²¹⁴ Pb	242	0.11534	0.04953	0.0471
²⁰⁸ TI	252.6	0.00179	0.00102	0.0010
²²⁷ Th	256.2	0.00487	0.00239	0.0016
²¹⁴ Pb/ ²²⁷ Ra	258.9	0.00774	0.00452	0.0028
²²⁸ Ac	270.2	0.04244	0.01631	0.0175
²¹⁴ Pb	274.8	0.00320	0.00193	0.0009
²⁰⁸ TI/ ²²⁷ Ra	277.4	0.01666	0.00858	0.0065
²¹² Bi	288.1	0.00193		0.0005
²¹⁴ Pb	295.2	0.21707	0.08974	0.0880
²¹² Pb/ ²²⁷ Ra	300.1	0.02595	0.00983	0.0099
²²⁷ Ra	302.7	0.00117		
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²¹⁴ Pb	323.8	0.00036		0.00064
²²⁸ Ac	327.6	0.02033	0.00889	0.00801
²²⁸ Ac	332.4	0.00193		
²²⁸ Ac	338.3	0.08217	0.03191	0.03145
²²⁸ Ac	341	0.00226		
²¹⁴ Pb	351.9	0.36334	0.15232	0.14524
²¹⁴ Bi	386.8	0.00237		0.00108
²¹⁴ Bi	388.9	0.00351		0.00134
²¹⁹ Rn	401.8	0.00343	0.00006	0.00137
²¹⁴ Bi	405.7			0.00121
²²⁸ Ac	409.5	0.01009	0.00424	0.00444
²¹⁴ Bi	426.5	0.00160		
⁴⁰ K (DEP)	438.8	0.00397	0.00153	0.00164
²¹² Bi	452.8	0.00189		
²¹⁴ Bi	454.8	0.00165		
²²⁸ Ac	463	0.02421	0.00968	0.00918
²¹⁴ Bi	474.4			0.00008
²¹⁴ Pb	480.4	0.00139		0.00094
²¹⁴ Pb /¹⁴⁰La	487.1	0.00410	0.00183	0.00125
²²⁸ Ac	503.8	0.00001		
²⁰⁸ TI	510.8	0.04917	0.02644	0.02596
²²⁸ Ac	546.5	0.00049		0.00053
²²⁸ Ac	562.5	0.00454	0.00119	0.00097
²¹⁴ Pb	580.1	0.00245		0.00061
²⁰⁸ TI	583.2	0.12447	0.04912	0.04890
²¹⁴ Bi	609.3	0.25491	0.10412	0.10216
²¹⁴ Bi	665.5	0.00643	0.00305	0.00307
²¹⁴ Bi	703.1	0.00282	0.00141	0.00095
²²⁸ Ac	707.4		0.00084	
²¹⁴ Bi	719.9	0.00110		0.00106
²¹² Bi	727.2	0.02645	0.01058	0.01012
²¹⁴ Bi(DEP)	742.5	0.00223	0.00038	
²²⁸ Ac	755.5	0.00422	0.00070	0.00175
²⁰⁸ TI	763.1	0.00165	0.00042	0.00082
²¹⁴ Bi	768.4	0.02236	0.00907	0.00950
²²⁸ Ac	772.3	0.00467	0.00154	0.00196
²²⁸ Ac	782.1	0.00151		
²¹² Bi/ ²¹⁴ Pb	785.4	0.00911	0.00373	0.00312
²²⁸ Ac	795	0.01395	0.00559	0.00536
²¹⁴ Bi	806.2	0.00490	0.00224	0.00198
²²⁸ Ac	830.5	0.00221		

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228	005 7	0.00533	0.00045	0.004.00
²²⁰ AC	835.7	0.00532	0.00245	0.00169
²¹⁴ PD	839	0.00513	0.00147	0.00163
	846.8	0.01011	0.00031	0.0050
200	860.6	0.01341	0.00595	0.00534
⁴⁰ SC	889.3		0.00034	0.00051
²²⁸ Ac	904.2	0.00190	0.00065	0.00080
228Ac	911.2	0.07908	0.03190	0.03131
²¹⁴ Bi	934.1	0.01140	0.00457	0.00452
²²⁸ Ac	964.8	0.01533	0.00642	0.00611
²²⁸ Ac	969	0.04651	0.01859	0.01828
^{234m} Pa	1001	0.00415	0.00158	0.00218
²¹⁴ Bi	1032.4	0.00062		
²¹⁴ Bi	1052	0.00120		0.00017
²²⁸ Ac	1165.2	0.00085		
²¹⁴ Bi	1070	0.00076	0.00029	0.00034
²¹² Bi	1078.6	0.00149	0.00012	0.00068
²²⁸ Ac	1110.6	0.00114		0.0007
⁴⁶ Sc/ ²¹⁴ Bi	1120.2	0.05102	0.02123	0.02036
²¹⁴ Bi	1155.2	0.00552	0.00211	0.00237
²¹⁴ Bi	1207.7	0.00113	0.00055	0.00027
²¹⁴ Bi	1238.1	0.01940	0.00779	0.00736
²²⁸ Ac	1247.1			0.00001
²¹⁴ Bi	1281	0.00418	0.00193	0.00137
²⁴ Na	1368.6		0.00478	
²¹⁴ Bi /⁵⁷Ni	1377.7	0.01198	0.00546	0.00517
²¹⁴ Bi	1385.3	0.00224	0.00076	0.00080
²¹⁴ Bi	1401.5	0.00382	0.00183	0.00152
²¹⁴ Bi	1408	0.00658	0.00283	0.00282
⁴⁰ K	1460.8	0.32417	0.13591	0.13247
²²⁸ Ac	1495.9	0.00175	0.00074	0.00059
²²⁸ Ac	1501.6	0.00074	0.00028	
²¹⁴ Bi	1509.2	0.00589	0.00256	0.00181
²¹⁴ Bi	1512.7	0.00077	0.00032	0.00000
⁴² K	1524.7		0.00108	0.00000
²¹⁴ Bi	1538.5	0.00111		0.00042
²¹⁴ Bi	1543.3	0.00083	0.00045	0.00049
²²⁸ Ac	1580.5	0.00113		0.0005
²¹⁴ Bi	1583.2	0.00176	0.00044	0.00078
²²⁸ Ac	1588.2	0.00605	0.00187	0.00249
	2000.2	0.00000	0.00107	0.00215
²⁰⁸ TI (DFP)	1592 5	0.00619	0.00196	0 00274







²¹⁴ Bi	1599.3	0.00070		0.00045
²¹² Bi	1620.5	0.00269	0.00109	0.00113
²²⁸ Ac	1625.1	0.00049		
²²⁸ Ac	1630.6	0.00303	0.00137	0.00103
²²⁸ Ac	1638.3	0.00073		0.00040
²¹⁴ Bi	1661.3	0.00244	0.00087	0.00097
²¹⁴ Bi	1684	0.00045		
²¹⁴ Bi	1729.6	0.00836	0.00388	0.00337
²¹⁴ Bi	1764.5	0.03941	0.01624	0.01574
²¹⁴ Bi	1838.4	0.00068		0.00030
²¹⁴ Bi	1847.4	0.00521	0.00218	0.00225
²¹⁴ Bi	1873.1			0.00013
²²⁸ Ac	1887.1			0.00004
²¹⁴ Bi	1890.3			0.00004
²¹⁴ Bi	1895.9		0.00010	
²²⁸ Ac	1955.9			0.00014
²¹⁴ Bi	2016.7	0.00013		
²¹⁴ Bi	2052.9		0.00013	
²⁰⁸ TI (SEP)	2103.5	0.00587	0.00261	0.00214
²¹⁴ Bi	2118.6	0.00235	0.00090	0.00100
²¹⁴ Bi	2204.2	0.01056	0.00437	0.00442
²¹⁴ Bi	2292.4	0.00071	0.00014	0.00018
²¹⁴ Bi	2447.9	0.00310	0.00156	0.00102
²⁰⁸ TI	2614.5	0.04015	0.01643	0.01647

Gamma-ray spectrum:



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Fig. 58 Gamma-ray spectrum of a rock sample (sample 11) 3.7 h after neutron activation.



Fig. 59 Gamma-ray spectrum of a rock sample (sample 11) 25.3 h after neutron activation.



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Fig. 60 Gamma-ray spectrum of a rock sample (sample 11) 67 h after neutron activation.



Fig. 61 Gamma-ray spectrum of a rock sample (sample 11) 113.9 h after neutron activation.



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Fig. 62 Gamma-ray spectrum of a rock sample (sample 11) 168.7 h after neutron activation.



5. Laboratory analyses of the sediment samples

Measurements of the concentration of radium and potassium radioisotopes in the sediment samples were performed in an external laboratory - "Low-level Activity Research Laboratory", Institute of Physics, the University of Silesia in Katowice, Poland.

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Description of the sites where sediment samples were taken:

Hall ID	Depth below surface	Sites	Method of the sampling	Name of the sample				
	[III w.e.]							
Localization 1				Sediment sample loc.1				
Localization 2				Sediment sample loc.2				
Localization 3			direct collection	Sediment sample loc.3				
Localization 4	80 (layer 310)	floor	into plastic	Sediment sample loc.4				
Localization 5			containers	Sediment sample loc.5				
Localization 8				Sediment sample loc.8				
Localization 9				Sediment sample loc.9				

Tab. 15 Description of the sites where sediment samples were collected.





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Fig. 63 (a) diagram of the place where the samples were taken (b) localization 1 - sediment sampling, (c) localization 2 - sediment sampling.









Fig. 64 (a) diagram of the place where the samples were taken (b) localization 3 - sediment sampling, (c) localization 4 sediment sampling, (d) localization 5 - sediment sampling.



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Fig. 65 (a) diagram of the place where the samples were taken (b) localization 8 - sediment sampling.

a. Radium and potassium concentration in sediment samples

• Information about measurement

The concentration of radium and potassium isotopes in the sediment samples was determined by gamma spectroscopy with the HPGe detector in a lead shield (*Fig. 66a*). The activity of ⁴⁰K was calculated directly from a single 1460.8 keV line. The activity of ²²⁶Ra was calculated as the weighted mean of the values obtained from the ²¹⁴Pb (295.2, 351.9 keV) and ²¹⁴Bi (609.3, 1120.3 keV) isotopes, while ²²⁸Ra activity was calculated from the gamma lines 338.3 keV and 911.1 keV originating from ²²⁸Ac decay. The total duration of a single measurement depended on the sample activity.

Description of the procedure performed before measurements and conditions during measurements (e.g. use of a Marinelli beaker; shielding of the detector; drying, crushing, grinding, mixing of rock; other relevant information): **Before the measurements, the sediment sample was**



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dried, crushed, ground, mixed and placed in a plastic container or in Marinelli beaker (Fig. 66b,c,d), which was then sealed and left for one month to achieve secular equilibrium in the thorium and uranium series. The grains diameter after crushing the sediment sample were less than 1 mm. Measurements were made in a shielded cover made of lead and copper.



Fig. 66 (a) HPGe detector with shielding, (b) sediment sample in plastic container and (c) Marinelli beaker, (d) sediment sample preparation steps (drying).

Hall ID (name of sample)	Measu- rement method	Equipment type	Detection relative efficiency [%]	Collection period	²²⁶ Ra	Average results [Bq/kg] ²²⁸ Ra	⁴⁰ K
Sediment sample loc. 1				4.1 days	92.9±3.4	270.5±4.8	41.3±1.5
Sediment sample loc. 2		ma tro- y	20	2.9 days	116.3±4.3	403.5±7.4	56.5±3.5
Sediment sample loc. 3	Gamma			4.1 days	32.3±0.3	257±11	211±13
Sediment sample loc. 4	spectro-			2.0 days	38.9±1.4	30.1±0.6	234.1±3.8
Sediment sample loc. 5	metry			4.1 days	20.3±0.7	30.1±0.6	223.8±3.5
Sediment sample loc. 8				2.2 days	209.5±7.6	262.0±4.9	65.6±3.9
Sediment sample loc. 9				2.1 days	58.1±2.1	39.8±0.9	486.1±8.7

Tab.	16 Results	of radium	and	potassium	concentration	in	sediment samples.

Gamma-ray spectra:



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Fig. 67 Gamma-ray spectrum of a sediment sample loc.1.





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Fig. 69 Gamma-ray spectrum of a sediment sample loc.3.



Fig. 70 Gamma-ray spectrum of a sediment sample loc.4.



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Fig. 71 Gamma-ray spectrum of a sediment sample loc.5.



Fig. 72 Gamma-ray spectrum of a sediment sample loc.8.



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